



Review of Euratom projects on design, safety assessment, R&D and licensing for ESNII/Gen-IV reactor systems

B. Hatala (VUJE), M. Ferreira (VTT), J.-L. Kloosterman (TU DELFT), K. Mikityuk (PSI), M. Šípová (CVŘ)

Introduction

European Sustainable Nuclear Industrial Initiative (ESNII) considers:

- Reference solution: Sodium Fast Reactor **ASTRID**;
- 1st alternative: Lead-cooled Fast Reactor **ALFRED** supported by LBE facility **MYRRHA**;
- 2nd alternative: Gas-cooled Fast Reactor **ALLEGRO**.

In addition the following Gen-IV systems are supported by Euratom:

- Gen-IV Molten Salt Fast Reactor MSFR (mentioned in SRA Annex as an attractive long-term option);
- Gen-IV Supercritical Water Cooled Reactor (SWCR);
- Gen-IV European Sodium Fast Reactor **ESFR**.













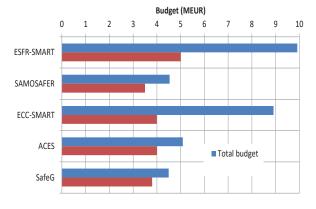


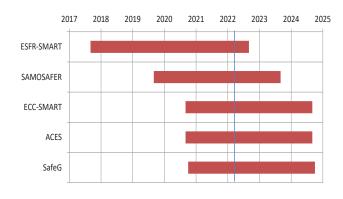






Four Generation-IV systems supported by the considered EU projects





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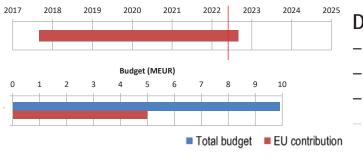




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1. ESFR-SMART: fact sheet

Name: European Sodium Fast Reactor Safety Measures Assessment and Research Tools



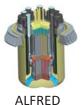
Domains:

- Design
- R&D
- Safety
- Licensino



Partners: 19 Countries: 9 Coordinator: PSI



















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1. ESFR-SMART: main goals

- Produce new experimental data to support calibration and validation of computational tools for each DiD level.
- Test and qualify new instrumentations to support their utilization in reactor protection system.
- Perform further calibration and validation of computational tools for each DiD level to support safety assessments of Gen-IV SFRs.
- Select, implement and assess new safety measures for commercial-size ESFR.
- Strengthen and link together new networks (sodium facilities and students).







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1. ESFR-SMART: selected results

Experimental programs:

CHUG: chugging boiling regime using steam-water rig

HAnSoLO: corium jet impingement using a water-ice system

JIMEC-I: ablation of a thick steel substrate with high temperature, high-velocity steel jet

- Benchmarking of codes:
 - Superphénix static and transient start-up tests
 - KNS-37 sodium boiling experiment
- Proposal of new safety measures
 - New core and system designs
- 1: Insulation with steel liner
- 2: Core catcher
- 4: Primary pump
- Above-core structure 6: Pit cooling system (DHRS-3)
- 7. Main vessel 8: Strongback 9: IHX
- 10: Reactor pit
- 11: Secondary sodium tank
- 12: Steam generator
- 13: Window for air circulation (DHRS-1)
- 14: Sodium-air HX (DHRS-1)
- 15: Air chimney (DHRS-1)
- 16: Secondary pump 17: Casing of SGs (DHRS-2)
- 18: Window for air circulation (DHRS-2)





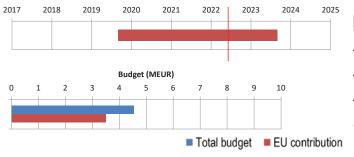




2. SAMOSAFER: fact sheet

Name: Severe Accident Modeling and Safety Assessment for Fluid-fuel Energy Reactors





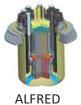
Domains:

- Design
- R&D
- Safety
- Licensing

Partners: 12 Countries: 7

Coordinator: TU DELFT





















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2. SAMOSAFER: main goals

Develop and demonstrate new safety barriers for more controlled behaviour of MSR in severe accidents, based on new simulation models and tools validated with experiments.

- Investigate and translate existing defence-in-depth safety approach to MSR
- Develop simulation code suite for neutronics, thermal hydraulics, thermo-physics modeling
- Develop and apply experimental setups for validation
- Design advanced barriers for severe accidents (freeze plugs, drain tanks, fission product extraction / immobilization)
- Update MSFR design

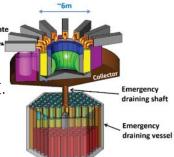






2. SAMOSAFER: selected results

- Specific MSR oriented defence in depth approach established by analysing safety functions
 of all fuel salt locations in reactor and by defining number of containment barriers.
- Thermochimica software coupled to JRC-Molten Salt Data Base for thermodynamic assessments of various salts.
- Molecular dynamics studies done on LiF-ThF4 using a new forced-field model.
- Preliminary scheme for reprocessing chloride salts developed.
- Salt freezing and re-melting modeling started.
- SIMMER code extended and prepared for Castillejos benchmark.
- SWATH-S facility extended to study radiation heat phenomena in salt.
- Design drawings of core and passive DHR system done.
- Summer school organized.





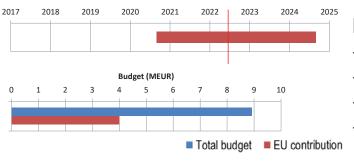




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3. ECC-SMART: fact sheet

Name: Joint European Canadian Chinese development of Small Modular Reactor Technology



Domains:

- Design
- R&D
- Safety
- Licensing



Partners: 16
Countries: 12

Coordinator: CV REZ

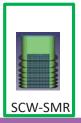




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3. ECC-SMART: main goal

Provide methodologies for safety evaluations and improvements for SCW-SMR, including experimental validation

- Assess corrosion behaviour of most promising candidates for SCW-SMR structural materials
- Define design requirements for SCW-SMR technology
- Develop and validate codes and assess proposed SCW-SMR concepts using these codes
- Provide reactor physics analysis of preliminary core layout
- Develop pre-licensing study and guidelines for safety demonstration







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3. ECC-SMART: selected results

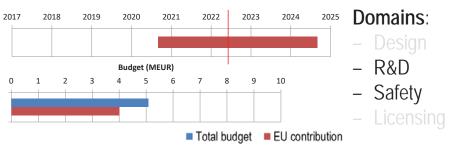
- Material testing: test matrix established with about 700 specimens:
 - Stainless steel 310S and alloy 800H selected as the most perspective material for fuel cladding;
 - experimental AFA (alumina forming austenitic alloy) supplied by China (USTB).
 - Most of specimens manufactured
- Innovative design of a small modular reactor cooled by SCW proposed based on HPLWR (high pressure light water reactor) using Canadian and Chinese experiences





4. ACES: fact sheet

Name: Towards improved assessment of safety performance for longterm operation of nuclear civil engineering structures





Partners: 10 Countries: 6 Coordinator: VTT



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4. ACES: main goals

- Improve understanding of ageing and deterioration of concrete for existing and nextgeneration NPPs
- Demonstrate and quantify inherent safety margins introduced by the conservative approaches used during design and defined by codes and standards
 - Assess corrosion of embedded liners in concrete
 - Characterise, predict and monitor ISR in concrete
 - Predict delayed strains of containment building
 - Assess performance of irradiated concrete





4. ACES: selected results

- Review of state-of-the-art of quantitative assessment of ageing of concrete SSC in NPPs.
- Improvement of phenomenological understanding and optimization of earlier detection of corrosion.
- Assessment of concrete structures affected by internal swelling reactions.
- Validation of existing constitutive laws and structural modelling approaches regarding the simulation of containment behaviour during operational phases.
- Generation of critical data currently missing from open literature on neutron-irradiation induced degradation of concrete aggregates relevant for European NPPs.







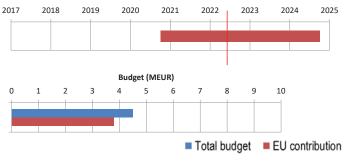
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5. SafeG: fact sheet

Name: Safety of GFR through innovative materials, technologies and processes





Domains:

- Design
- R&D
- Safety
- Licensing

Partners: 14 Countries: 7

Coordinator: VUJE













CW-SMR

MSFR







5. SafeG: main goal

- Continue development of ALLEGRO for demonstration of gas-cooled fast reactor technology
- Develop driver and refractory cores satisfying performance and safety requirements
- Assess materials with better performance for primary circuit
- Assess decay heat removal capabilities
- Fuel qualification







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5. SafeG: selected results

- Core safety significant progress beyond the state of the art of GFR core safety has already been made (start-up core optimization was completed). Further work will include optimization of reactivity feedback coefficients and irradiation capabilities of the ALLEGRO core designs.
- Automatic shutdown system Current design will be updated, using state-of-the art knowledge that is possessed by the consortium members who will work on this task.
- DHR system So far, decay heat removal for GFRs has been solved in a very similar way for all the reference concepts. Within SafeG, effort will be put into development of innovative DHR solution based on cutting-edge technology







Summary

- 5 EU project since 2017
- 4 ESNII/Gen-IV reactor systems
- 33 MEUR of total budget including 20 MEUR of Euratom contribution.
- 47 organizations from 19 countries
- Design, R&D, safety and licensing aspects







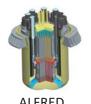


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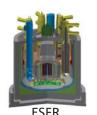
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